

Gen IV GRNS April 2-3; Washington, DC

Sustainability Scores

Report From Gen IV Fuel Cycle Crosscut Group (FCCG)



Conclusions: Sustainability Scoring

- The FCCG Believes the Scoring achieved Acceptable Consistency in Distinguishing Among Fuel Cycle Classes
 - Achieved Discrimination among the FCCG Generic Fuel Cycles
 - * Once-through/partial recycle; full fissile recycle; full actinide recycle
 - Where knowledge is lacking: achieved consistency in <u>relative</u> scores for distinguishing fuel cycle classes
 - * Contribution of "conditioning" to waste mass, waste volume
 - * Contribution of minor actinides to 500y heat load and toxicity source term
- Within a Fuel Cycle Class, Distinctions were Difficult to Score; and Several FCCG Consistency Ground Rules Affected Some Scores
 - Coarse Granularity of the SU-1 Metric (30% range of center box)
 - <u>All</u> Full recycle is 10X better than top box in SU1 little distinction
 - Thermal reactor Once-through vs MOX-mono vs DUPIC
 - * Difficult to distinguish by score within a 30% range of the same as Reference Box
- Nonproliferation Criteria are currently narrowly focused on Power Plant Link in the Fuel Cycle
 - Further developments needed to distinguish among and within fuel cycle classes

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Sustainability Criteria and Metrics

<u>Criterion</u>

<u>Reference</u>
<u>Scale</u>
<u>Granularity</u>

SU1 Tonnes of U Required/GWe y 150-200 linear coarse (30% range center to center)

Fully quantifiable by Formula [Assumes we count lifetime fissions – even with interim storage & even with mono or multi recycle in other reactors]

- Once-Through and Partial Recycle
 - Depends on
- * enrichment
- * enrichment of tails
- * Ave discharge burnup
- * station conversion efficiency
- For purpose of Consistency
 - * FCCG defined tails enrichment as 0.3%
 - * FCCG urged use of Once-Through unless strong case made for MOX or DUPIC
- Enrichment dependence dominates versus burnup and station efficiency effects
 - * Coarseness of Metric Scale: difficult to improve on reference

: easy to degrade using higher enrichment to increase burnup



Sustainability Criteria and Metrics

- SU1: Full Fissile and Full Actinides Recycle
 - Depends on

- * Number of Recycle Passes (1/atom percent burnup)
- * Loss to waste per Pass
- All Gen-4 Recycle concepts are full actinide recycle; all are ~10X better than top box
- Coarseness of metric: hard to fall below top box even with large recycle losses or low burnup

		<u>Reference</u>	<u>Scale</u>	<u>Granularity</u>
•	SU2-1 Tonnes of SNF or HLW/GWe y	15-20	Linear	Coarse
	Sent to Waste			(30% range center to center)

- Intent of EMG is Tonnes of <u>Conditioned</u> SNF or Tonnes of HLW <u>including diluent</u>
 - Data for many of Generation-4 concepts was not available to TWG's
- For Consistency of <u>relative</u> ratings among concepts
 - FCCG used (Mass of fission products + mass of heavy metal)
 - This surrogate metric is fully quantifiable by formula
 - ~ 1 gm/MW_{th}d * 1/η + Heavy Metal Sent to Waste in SNF or Lost in Recycle
- The Numerical SU1 and SU2-1 results were generated by FCCG by formula using TWG input
 - For SU1 Mass of virgin U/GWe y
 - For Su2-1 Mass of (FP+HM) to Waste/GWe y
 - They clearly distinguish recycle from once-through
 - Less clear once-through vs partial recycle



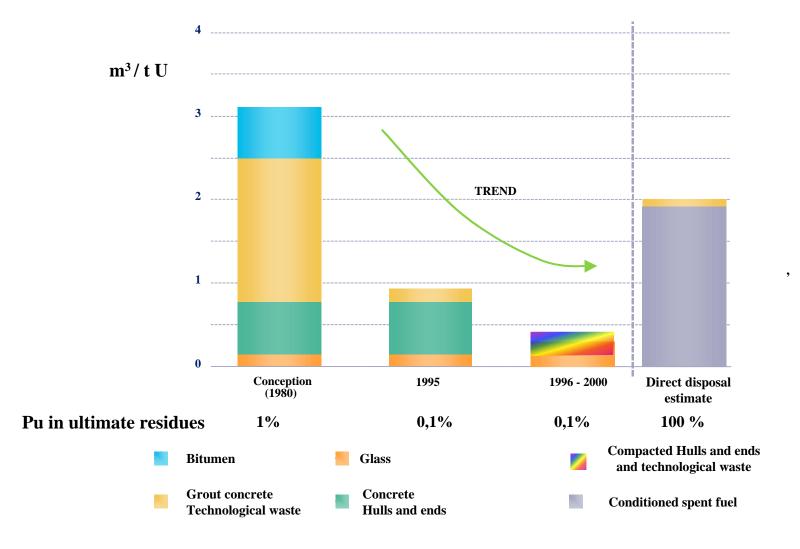
Sustainability Criteria & Metrics

<u>Reference</u> <u>Scale</u> <u>Granularity</u>

- SU2-1 Volume of SNF or HLW 15-20 M³/GWe y Linear Coarse (30% range center to center)
- The intent of EMG is volume of SNF and/or HLW as conditioned for disposal but not including any casks
 - The data were not available for many of the Generation-4 concepts
- To judge the <u>relative</u> consistency of concepts
 - The FCCG relied on
 - Data from French PUREX HLW production vs SNF
 - Data from Triple A studies of Waste Volumes predicted for advanced recycle (on the basis of significant but incomplete waste form development; fabrication; and leach testing and of flow sheet development and bench testing of recycle/refab)
- French data from PUREX shown next

Fuel Cycle CrossCut Group

Volumes of final residues conditioned in UP3 ENERATION IV (High level and long-lived waste after conditioning)





Sustainability Criteria & Metrics

Reference

Scale

Granularity

SU2-1 (Heat Load in SNF or HLW at 500y/GWe y

1-3 kw/GWe y

~linear Very coarse

(~ 70% center to center)

Toxicity Source Term in SNF or HLW at 500y per GWe y

500-1500 MSV/GWe y ~log

(~75%)

- At 500 y's both Heat load and Toxicity Source Term of SNF
 - Are dominated by the trivial weight fraction of transuranics in the SNF mass * i.e., ~ 1.5 w/o Pu + ~0.15 w/o Minor Actinides in SNF
 - The Fission Product Contributions are small by 500y
 - * FP dominates heat source at short time (principally Cs, Sr with ~35y half life) but have become secondary to actinides by 500y's
 - * FP's dominates toxicity source term at short time but have decayed to below that of actinides by 500v's
- The 500y Heat Load and Toxicity
 - Depend strongly on the (Minor Actinide)/(Total transuranic) mass fraction (Minor actinides heat load and toxicity per unit mass are high)

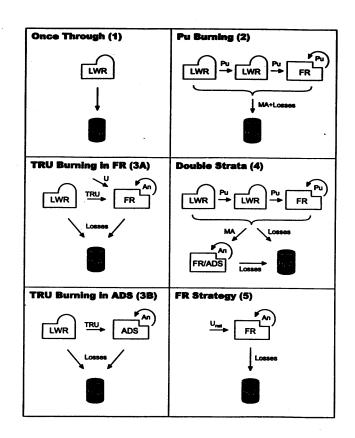


Sustainability Criteria & Metrics

- To Improve Performance against these criteria
 - a. Send less TRU to waste/GWe y
 - b. Send TRU of smaller minor actinide content to waste/GWe y
- For Once-Through and partial Recycle in Thermal reactors (where SNF goes to waste)
 - The TRU mass/GWe y (and heat load and toxicity source term)
 - * Decrease with station efficiency at any given burnup but not 70 to 75%
 - * Increase with burnup because MA fraction increases
- For Full Fissile or Full Actinide Recycle in Fast Reactors (where only trace losses go to waste)
 - The TRU mass/GWe y sent to waste is reduced vis-à-vis once-through
 - The MA mass/GWe y sent to waste depends on choice of full fissile vs full actinides recycle
- For most Generation-4 concepts, the TWG's had incomplete information
- To check scoring consistency, the FCCG relied on several previous studies (OECD-NEA, Triple A)
 - OECD-NEA result next

OECD-NEA Study Shows Importance of Minor Actinide Recycle vs Pu Recycle Only

1 Gwe y = 8.76 terrawatt hr



1.E+9 1600 Million Sv 1.E+8 GWe y 1.E+7 1.E+6 **9** 1.E+5 **X/X** 1.E+4 Once-Through --- Plutonium Burning - - TRU Burning in FR 1.E+3 - - TRU Burning in ADS 1.E+2 ---- Double Strata FR Strategy 1.E+1 1.E+0 1.E+1 1.E+2 1.E+3 1.E+4 1.E+5 1.E+0 1.E+6 1.E+7 Time after fuel reprocessing (a)

Fig. 1. Nuclear Fuel Cycle Schemes of the OECD-NEA Study

Fig. 2. Evolution of the Actinide Waste Radiotoxicity (Average Burnup of Metal and Nitride Fuel: 150 GWd/t^{HM})

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SU2 Summary Observations Recycle Long Term Toxicity, Heat Load and HLW Waste Volume vs SNF

- Heat & Toxicity are dominated by a trivial mass fraction of SNF Pu ~ 1.5 w/o
 Ma ~ 0.15 w/o
- Any fast multi recycle of all TRU puts you in the top box for mass to waste (by ~ factors of 10) – even poor recycle recovery fractions
- This will reduce <u>decay heat</u> (long term) by factor of 50 to >100 This will reduce <u>toxicity</u> (long term) by factor of 100 to 200

However

- Multi Recycle of Pu only (and MA go to waste)
 - Gains < factor of 10 (not several hundred) on toxicity
 - Recycling MA → Remote Fabrication!
- <u>Volume</u> of HLW forms from Multi TRU Recycle are factors of 2 to 3 larger than SNF Volume; MOX Mono Recycle <1/2 (but MOX SNF still has to be disposed)

Sustainability Criteria and Metrics

- SU3-1 Avoid Separated Weapons Usable Materials
 - SU3-2 Impede Handling and Recovery of weapons Useable Material
 - SU3-3 Reactors Have Passive features That Resist Sabotage
- EMG intent
 - Focus on the Power Plant itself
 - Defer a cradle to grave fuel cycle nonproliferation evaluation
 - Until the fuel cycle technologies and facility designs of the leading concepts are better defined
 - Until an evaluation approach for Generation-4 is better defined
- These criteria and metrics are clearly stated; don't require judgments
 - There were no consistency issues here
- Observation for the future extension of Consideration to Cradle to Grave Fuel Cycle
 - Every full recycle concept in Generation-4 employs the same strategies
 - * Full actinide recycle in commixed product streams
 - * Remote refabrication
 - * Send only trace losses of fissile to waste; ~fissile free repository
 - * Reduce enrichment deployments

Therefore

 Distinguishing among them will require accounting for cost effectiveness of proliferation resistance as well as technology per se



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